

**한국표준형 원전의 안전주입계통 설계최적화**

**Optimization of Safety Injection System Design  
for Korean Standard Nuclear Power Plant**

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**요 약**

한국표준형 원전의 안전주입계통의 설계최적화를 위하여 인허가된 해석방법론을 적용하여 사 고별로 안전주입계통 주요기기인 고압안전주입펌프, 저압안전주입펌프 및 안전주입탱크의 용량을 각각의 변수로 하여 일련의 범위해석을 수행하였다. 해석결과, 고압안전주입펌프 관련해서는 가압 기에 3-train POSRV 계통을 채택하고, 저압안전주입펌프 관련해서는 정지냉각계통의 열교환기 용량증대 또는 NSSS 설계요건 완화를 병행할 경우, 원전의 안전성관점에서 안전주입계통 주요기 기의 용량은 표준원전인 울진3,4호기의 설계용량 대비 75%로 축소설계가 가능한 것으로 평가되었 다.

**Abstract**

A series of scoping analyses have been performed, using safety analyses methodology approved from regulatory authorities, for the safety injection system (SIS) of Korean standard nuclear power plant (KSNP) to optimize the designs of the high pressure safety injection pump (HPSIP), the low pressure safety injection pump (LPSIP) and the safety injection tank (SIT). The result of these analyses provides the possibility of reducing the SIS down 75% of current capacity subject to design changes such as adopting a 3-train power operated safety

relief valve (POSRV) system on the pressurizer for HPSIP, and increasing capacity of the heat exchanger of the shutdown cooling system (SCS) or relaxing the design requirements of nuclear steam supply system (NSSS) for LPSIP.

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**Evaluation of Analytically Scaled Model for Small Break Loss of Coolant Accident at  
Low Power**

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Abstract

The present paper performs analytical evaluations for the potential distortions caused by the scaled models using RELAP5/MOD3 computer codes. By use of scaling analysis, two scaled models with same volumetric ratio are constructed for Korean Next Generation Reactor (KNGR), which is an advanced light water reactor. The scaling methodology adopted in this paper preserves two-phase natural circulation similarities between prototype and scaled models. One scaled model is at full height with reduced flow area. The other model is at reduced height with reduced flow area. By using appropriate scale factors the RELAP5/MOD3 input models are developed. Then, the transient responses of two ideal scaled models are simulated for Small Break Loss of Coolant Accident (SBLOCA) by using RELAP5/MOD3 computer code. The transient responses of two scaled models are compared with those of the prototype. The results indicate that qualitative and quantitative similarities are well preserved for both models during SBLOCA with different break sizes.