

Vitrification of the Spent Nuclear Fuel Using Iron Phosphate Glasses

Cheong Won Lee*, Sung Gyun Shin, Yong Uk Kye, and Jong Heo

Pohang University of Science and Technology, 77, Cheonam-ro, Nam-gu, Pohang-si, Gyeongsanbuk-do, Republic of Korea

*schwarz@postech.ac.kr

1. Introduction

Average amount of the spent nuclear fuel (SNF) being discharged from the nuclear power plants (NPP) in Korea reached 760 MTU per year. They have been stored in the spent fuel pool inside the NPP and the storages will be saturated by 2024 [1]. Direct disposal inside the deep geological repository has been considered for a safe disposal of SNF. We propose the direct vitrification using phosphate glass can reduce the amount of SNF that needs to be stored inside the repository.

2. Experimental procedure

2.1 Preparation of the glasses

The nominal compositions of the iron phosphate glasses are given in Table 1. Starting powders of $\text{NH}_4\text{H}_2\text{PO}_4$, Fe_2O_3 and Al_2O_3 were mixed and CeO_2 powders were used as a surrogate for UO_2 . Batch mixtures were calcined in alumina crucibles at 220°C for 1 hour and melted at 1300°C for 1 hour. Melts were then quenched by pouring and pressing between two brass molds in the air to form glasses black in color.

Table 1. Nominal and analyzed composition of the iron phosphate glass prepared

Elements	Nominal composition		Analyzed composition (wt.%)
	(mol%)	(wt.%)	
P_2O_5	60	57.30	55.45
Fe_2O_3	15	16.12	18.43
Al_2O_3	5	3.43	4.20
CeO_2 (UO_2)	20	23.16 (32.10)	21.92 (30.38)
Total	100.00	100.00	100.00

2.2 Properties analysis

X-ray fluorescence (XRF) spectroscopy was used to analyze the glass composition quantitatively. Density was measured at room temperature by the Archimedes method using deionized water as a medium. Glass transition temperature (T_g) and specific heat (C_p) were determined using a differential scanning calorimetry (DSC) at a heating rate of $10^\circ\text{C}/\text{min}$ and thermal conductivity (k) was measured using laser flash method (LFA).

Chemical durability of the glasses were evaluated using the product consistency test (PCT) [2]. Glasses were crushed and sieved to $75 \sim 150 \mu\text{m}$ in size. Glass powders were then washed with deionized water and ethanol and 1.5 g of powders were soaked in 15 mL of deionized water inside a Teflon vessel and kept at $90 \pm 2^\circ\text{C}$ for 7 days. The leachate was filtered using syringe with a $0.45 \mu\text{m}$ filter. Concentrations of elements in the leachate were analyzed using inductively coupled plasma mass spectrometer (ICP-MS).

Assessment of nuclear criticality safety and the stabilities of the glasses against the heat was performed using Monte Carlo N-particle 6 (MCNP 6) and computational fluid dynamics (CFD).

3. Results

3.1 General properties

Compositions of the glasses analyzed were similar to the nominal composition without any significant loss of individual components (Table 1). The glass contains 21.92wt.% of CeO_2 . If UO_2 were used, concentration should be 30.38wt.%. Normalized elemental release of Ce after PCT was $2.3 \times 10^{-4} \text{ g}/\text{m}^2$ and all other elements were $< 0.2 \text{ g}/\text{m}^2$ significantly below US regulation of $2 \text{ g}/\text{m}^2$. The iron phosphate

glasses have $T_g = 540 \pm 2^\circ\text{C}$, $C_p = 320.0 \pm 15 \text{ J/kg } ^\circ\text{C}$ and $k = 0.33 \pm 0.015 \text{ W/m K}$ with a density of 3.15 g/cm^3

3.2 Evaluation of the vitrified SNF

Values of effective multiplication factor (k_{eff}) were 0.495 and 0.755 when the canisters were in the normal state and accident case, respectively. Both values were much lower than regulation of 0.95. It indicated that vitrified SNF has enough nuclear criticality safety for a disposal inside the repository.

Temperature of the buffer in repository for vitrified SNF reaches as high as 82.23°C when a spacing between canisters is 2m and distance between disposal tunnels is 10m. Area efficiency of the vitrified SNF disposal is almost 7 times higher than the case of the direct disposal [3]. In addition, the highest temperature expected at the center of the glass is 82.73°C and it is considerably below the glass transition temperature of the glass prepared (540°C).

4. Conclusion

Iron phosphate glasses containing CeO_2 , as a surrogate for UO_2 , were developed as a potential waste form for immobilization of SNF. The waste loading of CeO_2 (UO_2) in the glass was 21.92wt.% (30.38wt.%). Normalized elemental releases of all elements were $< 0.2 \text{ g/m}^2$, well below the US regulations. k_{eff} (0.495 and 0.755) were below the criticality safety regulations (0.95). Glass transition temperature (540°C) of the iron phosphate glass is much higher than the peak temperature of the vitrified SNF (82.73°C) inside the repository, thus providing the thermal stability of the waste forms.

ACKNOWLEDGEMENT

This research was supported by Basic Science Research Program through the National Research Foundation of Korea (NRF) funded by the Ministry of Science and ICT (NRF-2017R1A2B4006754)

REFERENCES

- [1] B.H. Park, "Assessment of spent nuclear fuel amounts to be managed based on disposal option in Republic of Korea", *Annals of Nuclear Energy*, 109, 199-207 (2017).
- [2] The Product Consistency Test (PCT), C 1285-02, ASTM International, West Conshohocken, PA, 2008 <www.astm.org>.
- [3] W. J. Cho, J. S. Kim, H. J. Choi, "Hydrothermal modeling for the efficient design of thermal loading in a nuclear waste repository", *Nuclear Engineering and Design*, 276, 241-248 (2014).